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- (3) Potential change in the risk to the public from the accidental off-site release of radioactive material;
- (4) Potential impact on radiological exposure of facility employees;
- (5) Installation and continuing costs associated with the backfit, including the cost of facility downtime or the cost of construction delay;
- (6) The potential safety impact of changes in plant or operational complexity, including the relationship to proposed and existing regulatory requirements:
- (7) The estimated resource burden on the NRC associated with the proposed backfit and the availability of such resources:
- (8) The potential impact of differences in facility type, design or age on the relevancy and practicality of the proposed backfit:
- (9) Whether the proposed backfit is interim or final and, if interim, the justification for imposing the proposed backfit on an interim basis.
- (d) No licensing action will be withheld during the pendency of backfit analyses required by the Commission's rules.
- (e) The Executive Director for Operations shall be responsible for implementation of this section, and all analyses required by this section shall be approved by the Executive Director for Operations or his designee.

[53 FR 20610, June 6, 1988, as amended at 54 FR 15398, Apr. 18, 1989; 72 FR 49504, Aug. 28, 20071

ENFORCEMENT

§ 50.110 Violations.

- (a) The Commission may obtain an injunction or other court order to prevent a violation of the provisions of—
- (1) The Atomic Energy Act of 1954, as amended;
- (2) Title II of the Energy Reorganization Act of 1974, as amended; or
- (3) A regulation or order issued pursuant to those Acts.
- (b) The Commission may obtain a court order for the payment of a civil penalty imposed under Section 234 of the Atomic Energy Act:
 - (1) For violations of—

- (i) Sections 53, 57, 62, 63, 81, 82, 101, 103, 104, 107, or 109 of the Atomic Energy Act of 1954, as amended;
- (ii) Section 206 of the Energy Reorganization Act;
- (iii) Any rule, regulation, or order issued pursuant to the sections specified in paragraph (b)(1)(i) of this section:
- (iv) Any term, condition, or limitation of any license issued under the sections specified in paragraph (b)(1)(i) of this section.
- (2) For any violation for which a license may be revoked under section 186 of the Atomic Energy Act of 1954, as amended.

[57 FR 55075, Nov. 24, 1992]

§ 50.111 Criminal penalties.

- (a) Section 223 of the Atomic Energy Act of 1954, as amended, provides for criminal sanctions for willful violation of, attempted violation of, or conspiracy to violate, any regulation issued under sections 161b, 161i, or 1610 of the Act. For purposes of section 223, all the regulations in part 50 are issued under one or more of sections 161b, 161i, or 1610, except for the sections listed in paragraph (b) of this section.
- (b) The regulations in 10 CFR part 50 that are not issued under sections 161b, 161i, or 161o for the purposes of section 223 are as follows: §§ 50.1, 50.2, 50.3, 50.4, 50.8, 50.11, 50.12, 50.13, 50.20, 50.21, 50.22, 50.23, 50.30, 50.31, 50.32, 50.33, 50.34a, 50.35, 50.36b, 50.37, 50.38, 50.39, 50.40, 50.41, 50.42, 50.43, 50.45, 50.50, 50.51, 50.52, 50.53, 50.56, 50.57, 50.58, 50.81, 50.90, 50.91, 50.92, 50.100, 50.101, 50.102, 50.103, 50.109, 50.110, 50.111.

[57 FR 55075, Nov. 24, 1992, as amended at 61 FR 39303. July 29, 1996]

ADDITIONAL STANDARDS FOR LICENSES, CERTIFICATIONS, AND REGULATORY AP-PROVALS

§ 50.120 Training and qualification of nuclear power plant personnel.

(a) Applicability. The requirements of this section apply to each applicant for and each holder of an operating license issued under this part and each holder of a combined license issued under part 52 of this chapter for a nuclear power plant of the type specified in §50.21(b) or §50.22.

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- (b) Requirements. (1)(i) Each nuclear power plant operating license applicant, by 18 months prior to fuel load, and each holder of an operating license shall establish, implement, and maintain a training program that meets the requirements of paragraphs (b)(2) and (b)(3) of this section.
- (ii) Each holder of a combined license shall establish, implement, and maintain the training program that meets the requirements of paragraphs (b)(2) and (b)(3) of this section, as described in the final safety analysis report no later than 18 months before the scheduled date for initial loading of fuel.
- (2) The training program must be derived from a systems approach to training as defined in 10 CFR 55.4, and must provide for the training and qualification of the following categories of nuclear power plant personnel:
 - (i) Non-licensed operator.
 - (ii) Shift supervisor.
 - (iii) Shift technical advisor.
- (iv) Instrument and control technician.
- (v) Electrical maintenance personnel.(vi) Mechanical maintenance per-
- sonnel.
 (vii) Radiological protection technician.
 - (viii) Chemistry technician.
 - (ix) Engineering support personnel.
- (3) The training program must incorporate the instructional requirements necessary to provide qualified personnel to operate and maintain the facility in a safe manner in all modes of operation. The training program must be developed to be in compliance with the facility license, including all technical specifications and applicable regulations. The training program must be periodically evaluated and revised as appropriate to reflect industry experience as well as changes to the facility, procedures, regulations, and quality assurance requirements. The training program must be periodically reviewed by licensee management for effectiveness. Sufficient records must be maintained by the licensee to maintain program integrity and kept available for NRC inspection to verify the adequacy of the program.

[72 FR 49505, Aug. 28, 2007]

§50.150 Aircraft impact assessment.

- (a) Assessment requirements. (1) Assessment. Each applicant listed in paragraph (a)(3) shall perform a design-specific assessment of the effects on the facility of the impact of a large, commercial aircraft. Using realistic analyses, the applicant shall identify and incorporate into the design those design features and functional capabilities to show that, with reduced use of operator actions:
- (i) The reactor core remains cooled, or the containment remains intact; and
- (ii) spent fuel cooling or spent fuel pool integrity is maintained.
- (2) Aircraft impact characteristics.¹ The assessment must be based on the beyond-design-basis impact of a large, commercial aircraft used for long distance flights in the United States, with aviation fuel loading typically used in such flights, and an impact speed and angle of impact considering the ability of both experienced and inexperienced pilots to control large, commercial aircraft at the low altitude representative of a nuclear power plant's low profile.
- (3) *Applicability*. The requirements of paragraphs (a)(1) and (a)(2) of this section apply to applicants for:
- (i) Construction permits for nuclear power reactors issued under this part after July 13, 2009;
- (ii) Operating licenses for nuclear power reactors issued under this part for which a construction permit was issued after July 13, 2009;
- (iii)(A) Standard design certifications issued under part 52 of this chapter after July 13, 2009;
- (B) Renewal of standard design certifications in effect on July 13, 2009 which have not been amended to comply with the requirements of this section by the time of application for renewal:
- (iv) Standard design approvals issued under part 52 of this chapter after July 13, 2009;
- (v) Combined licenses issued under part 52 of this chapter that:

¹Changes to the detailed parameters on aircraft impact characteristics set forth in guidance shall be approved by the Commission